

# EURISOL targetry challenges

EURISOL shall deliver beams of 3 orders of magnitude higher intensity than 1999 yields.

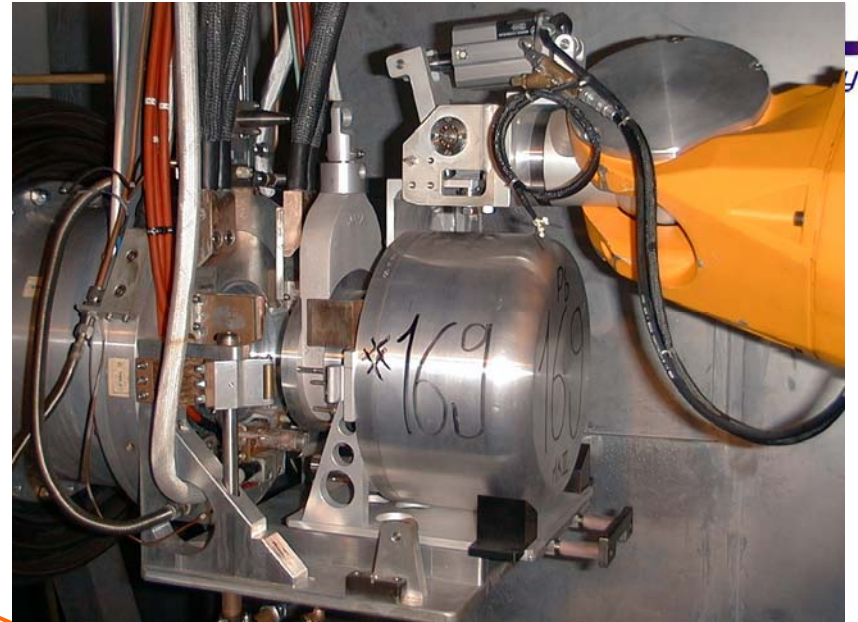
- Design of “oven + heat-transport system” target containers dissipating a direct GeV p-beam dc-power of > 100 kW to produce spallation generated RIBs.
- Computational optimization of the wall material and target geometry of a chosen element from a “*target vessel, transfer line & ion-source*” system to improve the release efficiency.
- Improve ion-source efficiencies and beam purity.
- Design high power density spallation n-sources (10 MW/l)
- Design actinide targets intercepting efficiently the neutrons generated by high power density n- sources.

# Introduction to ISOL RIB facilities

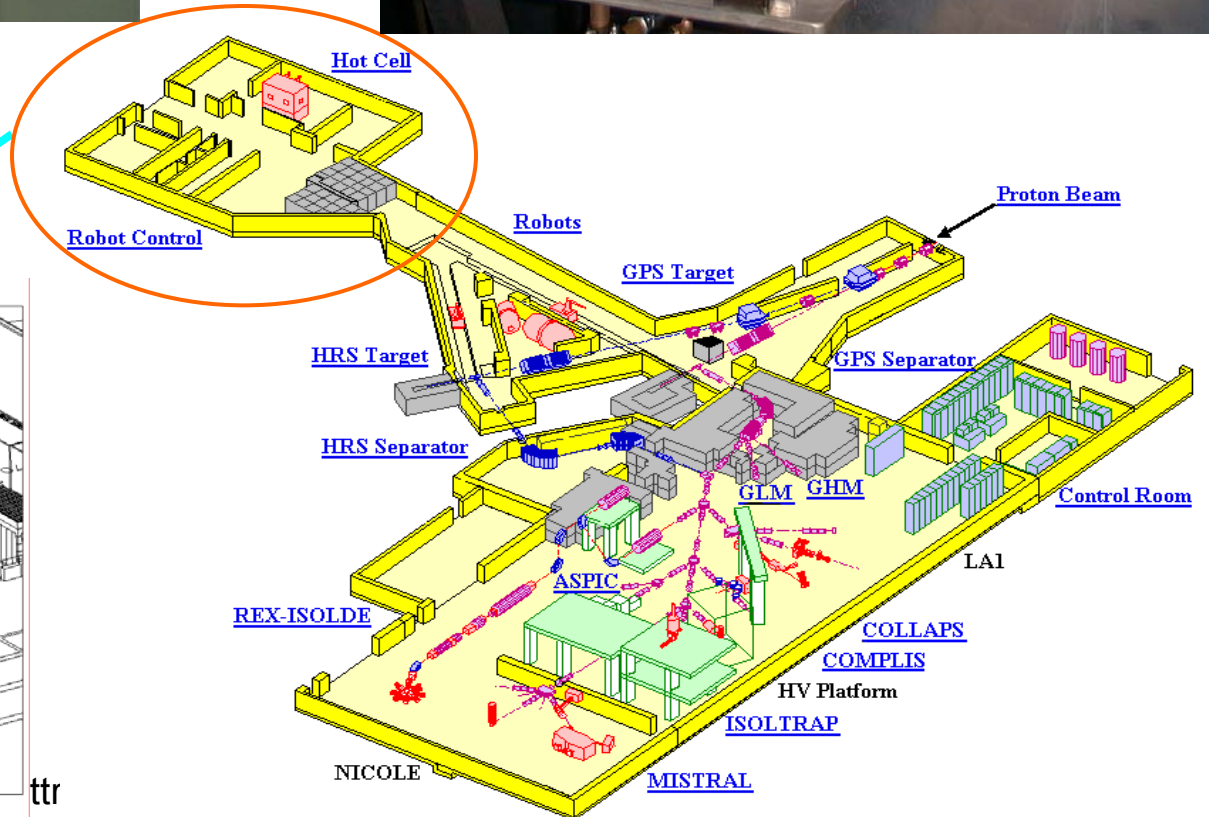
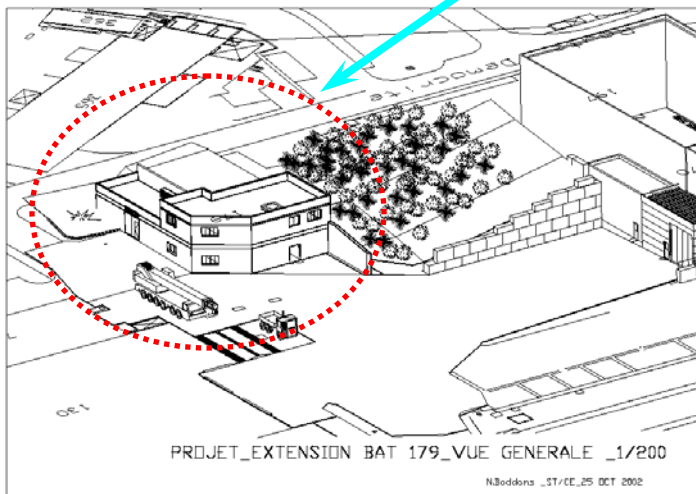
## Isotope **S**eparation **O**n **L**ine

- ISOLDE
- TRIUMF

# ISOLDE target handling.



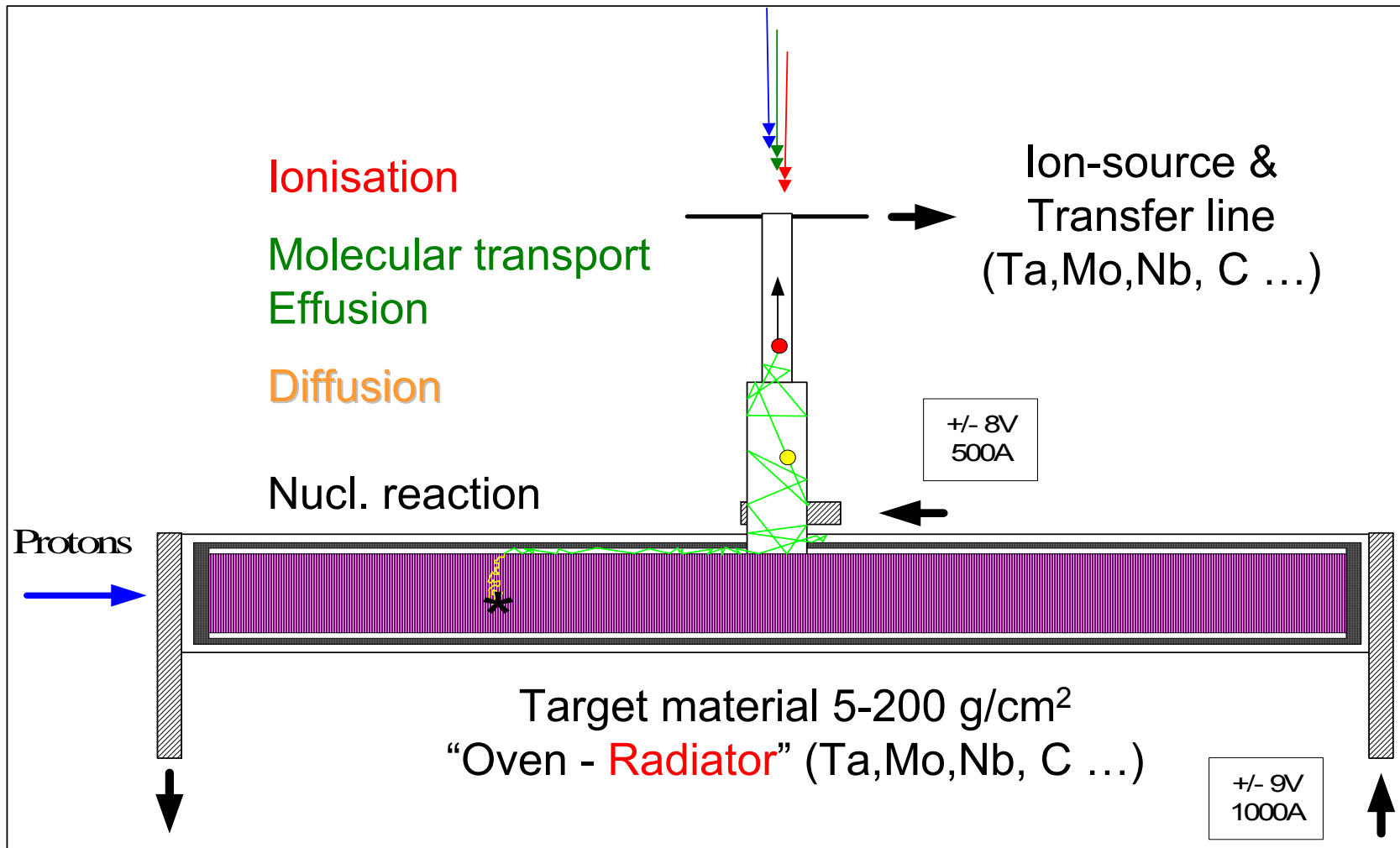
Class A laboratory  
 $\Sigma_{Isotopes} (Activity/LA) > 10'000$



# ISOL "Thick" targets

Pre-Separation, Separation  
Extraction, beam optics

(1 GeV ~ 200 g/cm<sup>2</sup>)



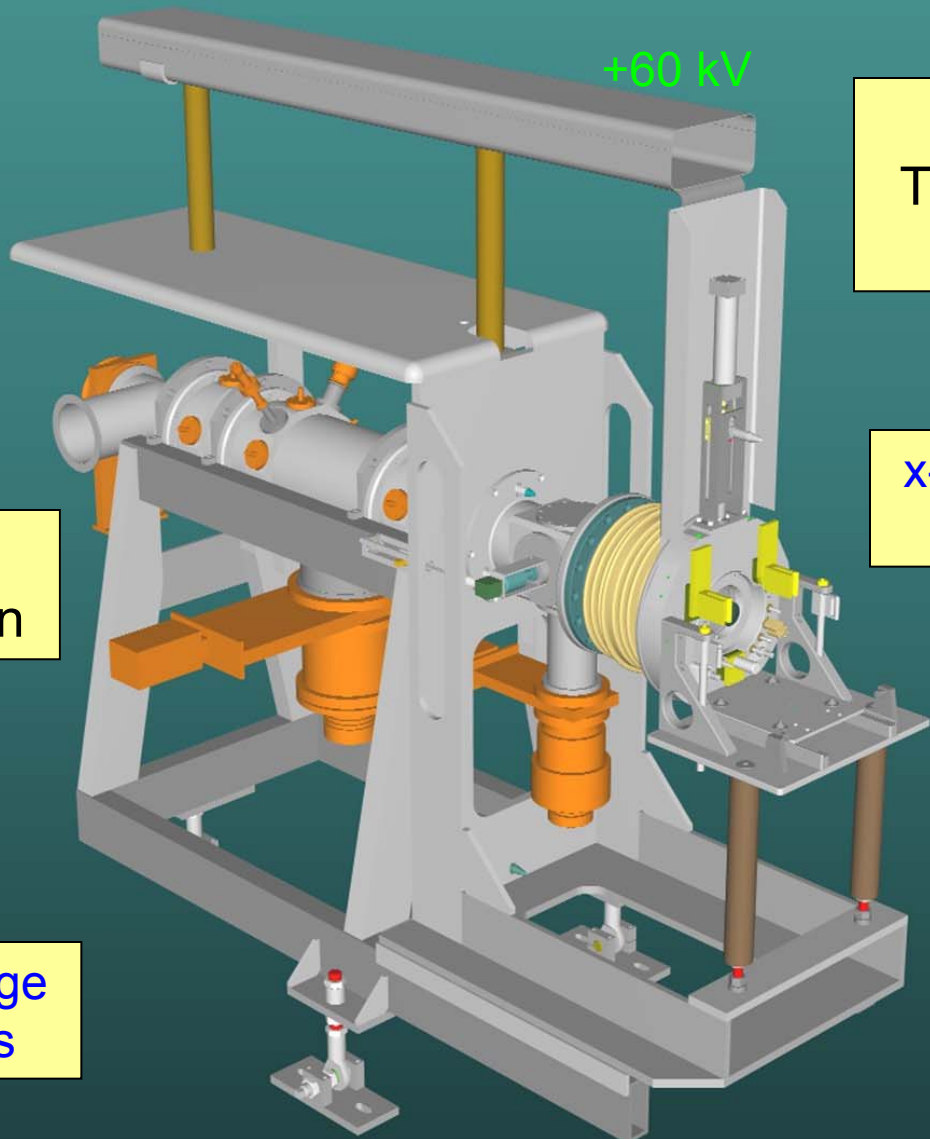
# ISOLDE Front-End *Technical design*

Strategy: 2 FE + 1 spare  
Production needs: 1 FE/3years  
"Minor" repair after 1 year of cooling

$5 \times 10^{19}$  p/y  
5 MGy/y

Faraday cup &  
QP triplet Section

Quick exchange  
of spare parts



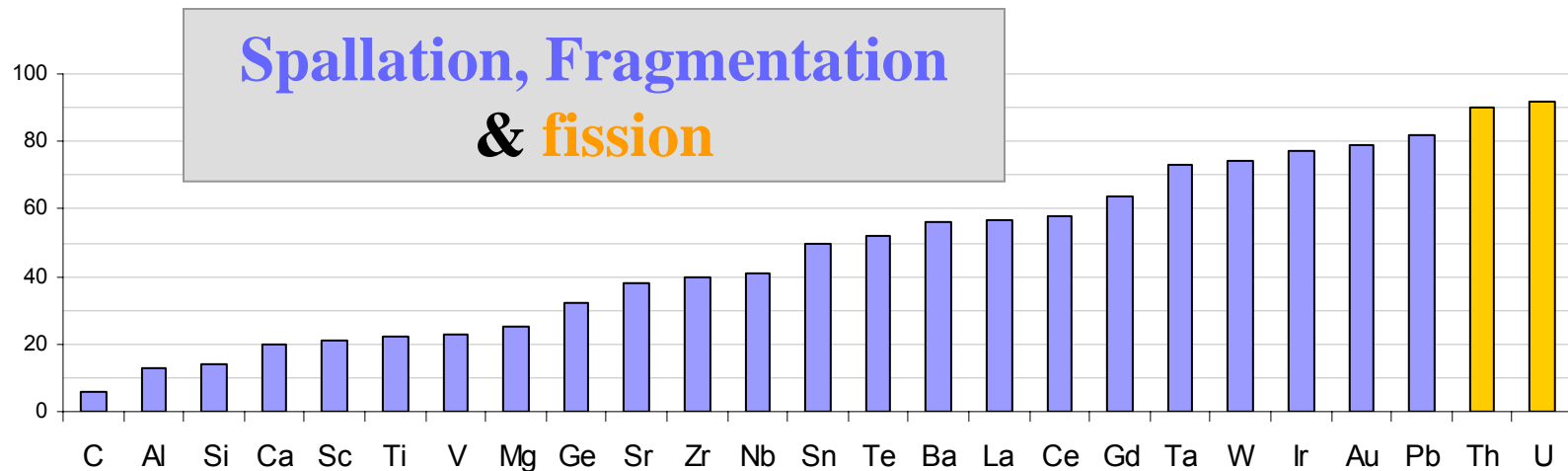
Exchangeable  
Target and Ion-Source  
coupling section

x-y Electrostatic steering  
+ z Movement

External alignment

S. Marzari  
& ATB-IF

# 6 ≤ Z ≤ 92 Target materials



25 Elements,  
~ 40 compounds:

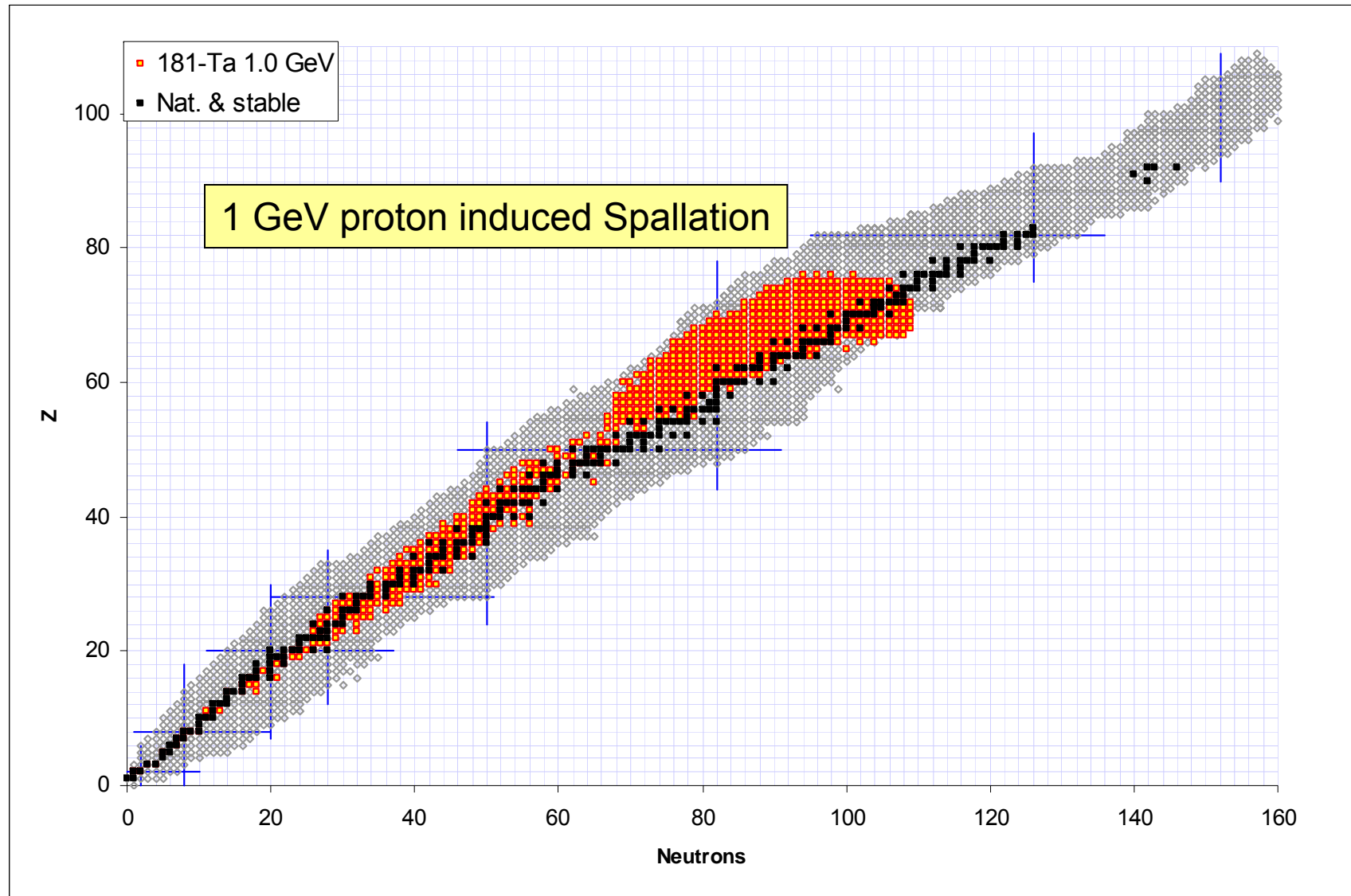
Oxides, Carbides  
Molten metals, - salts  
Thin foils, Powders

Target thickness: 4-220 g/cm<sup>2</sup>  
Cross section  $\sigma(E_{\text{proton}})$

Beam power for ISOL-RIB production:  
Liquids (1-2 kW), Solids (5 kW),  
Ta, Nb, SiC (20-30 kW @ TRIUMF)



# Ta + 1 GeV p, CASCABLA, A. Junghans



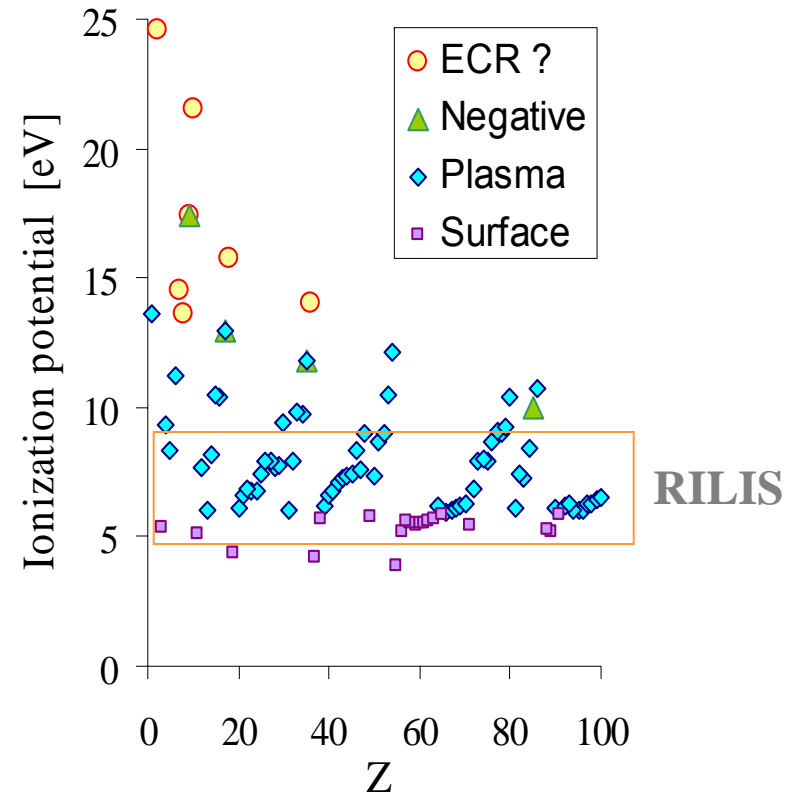
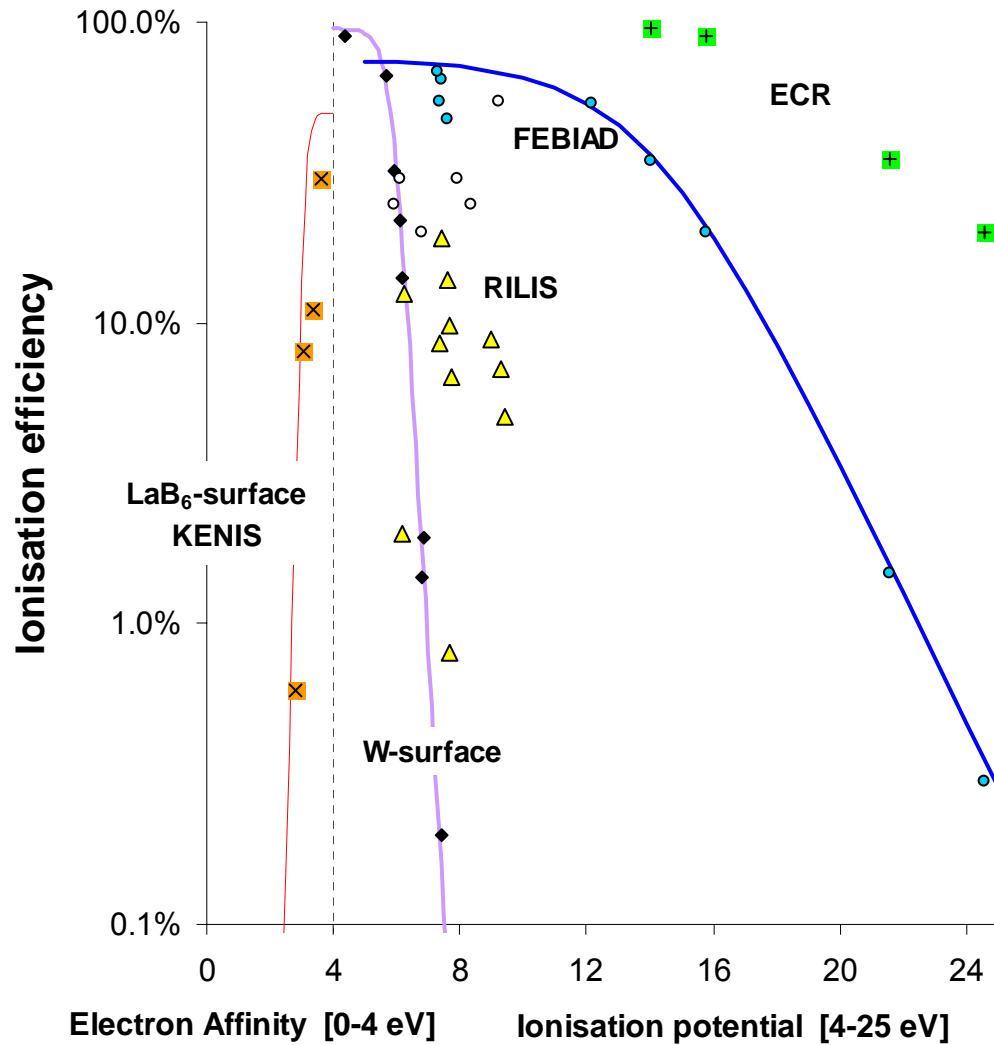
# Chemically selective RIB Ion-sources

- Surface ion-source
- Negative ion-sources
- FEBIAD ion-sources
- RILIS, Elements available 2004
- ECR 1+

1	H																	2	He																
3	Li	4	Be													5	B	6	C	7	N	8	O	9	F	10	Ne								
			7.0%																																
11	Na	12	Mg													13	Al	14	Si	15	P	16	S	17	Cl	18	Ar								
			9.8%																																
19	K	20	Ca	21	Sc	22	Ti	23	V	24	Cr	25	Mn	26	Fe	27	Co	28	Ni	29	Cu	30	Zn	31	Ga	32	Ge	33	As	34	Se	35	Br	36	Kr
											19.2%							0.8%		6.6%	4.9%														
37	Rb	38	Sr	39	Y	40	Zr	41	Nb	42	Mo	43	Tc	44	Ru	45	Rh	46	Pd	47	Ag	48	Cd	49	In	50	Sn	51	Sb	52	Te	53	I	54	Xe
																				14.0%	8.8%		8.5%												
55	Cs	56	Ba	57	La	72	Hf	73	Ta	74	W	75	Re	76	Os	77	Ir	78	Pt	79	Au	80	Hg	81	Tl	82	Pb	83	Bi	84	Po	85	At	86	Rn
87	Fr	88	Ra	89	Ac	104		105		106		107		108		109																			
57	La	58	Ce	59	Pr	60	Nd	61	Pm	62	Sm	63	Eu	64	Gd	65	Tb	66	Dy	67	Ho	68	Er	69	Tm	70	Yb	71	Lu						
																									2.0%	12.5%									
89	Ac	90	Th	91	Pa	92	U	93	Np	94	Pu	95	Am	96	Cm	97	Bk	98	Cf	99	Es	100	Fm	101	Md	102	No	103	Lw						

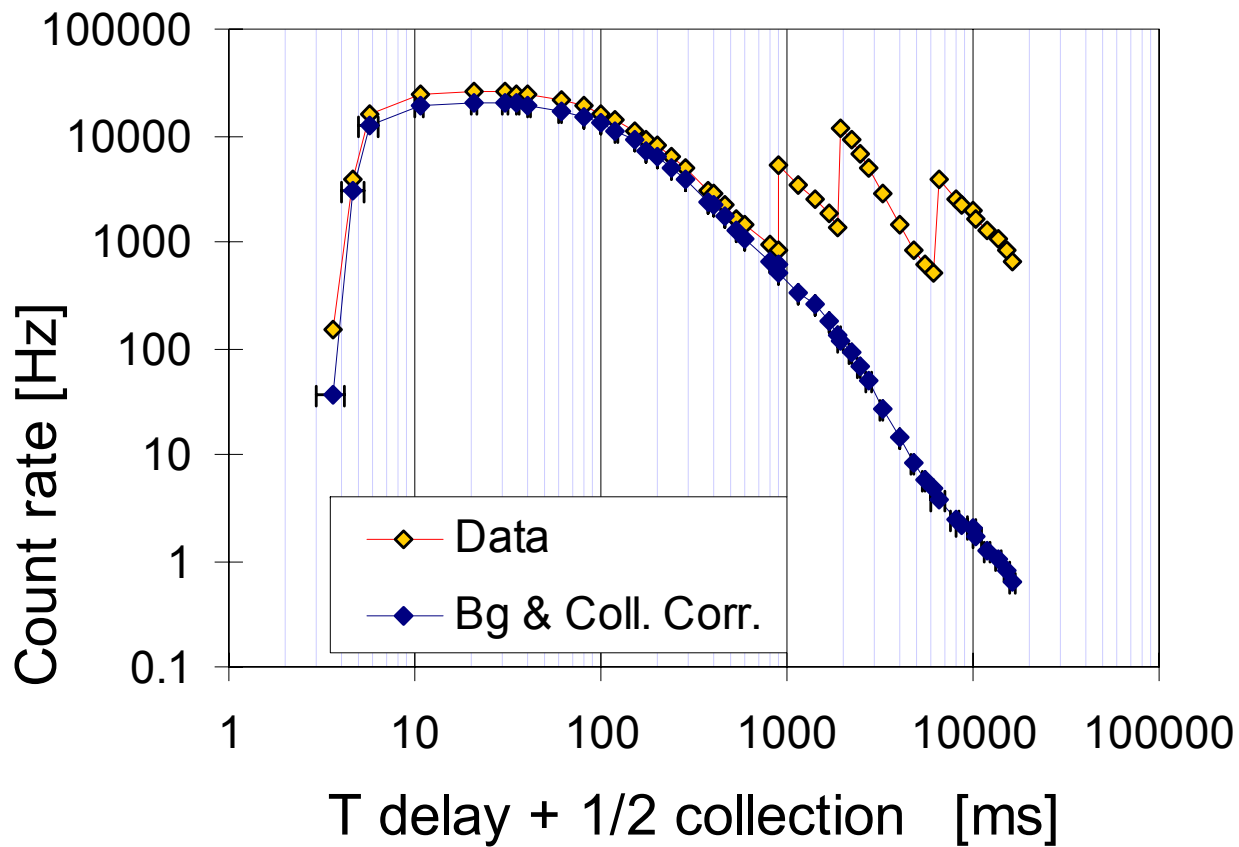


# Ion-sources efficiencies



# Release efficiency

effusion, diffusion

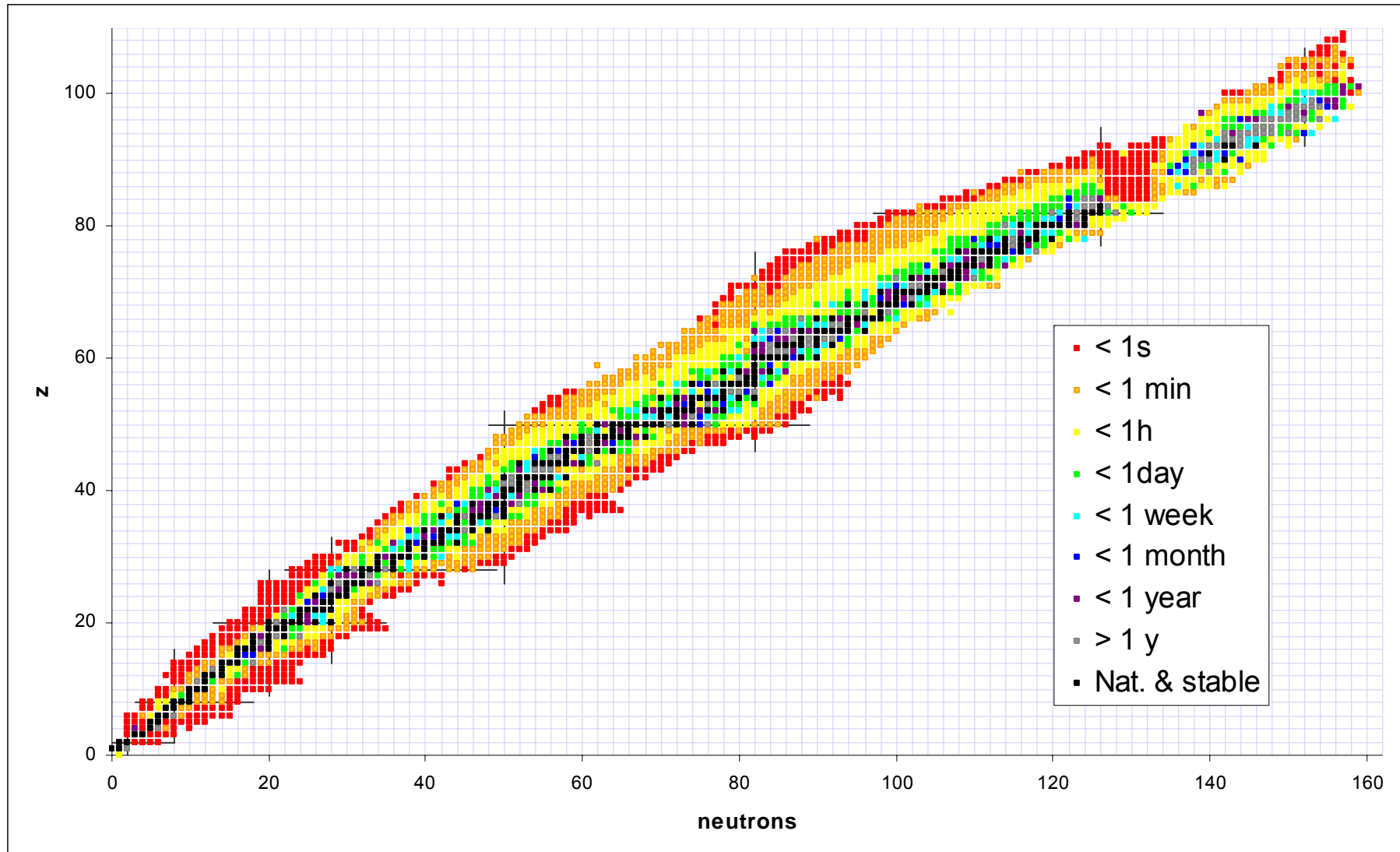


$^{25}\text{Na}$  (59.6s)  
 UC-118 2100°C

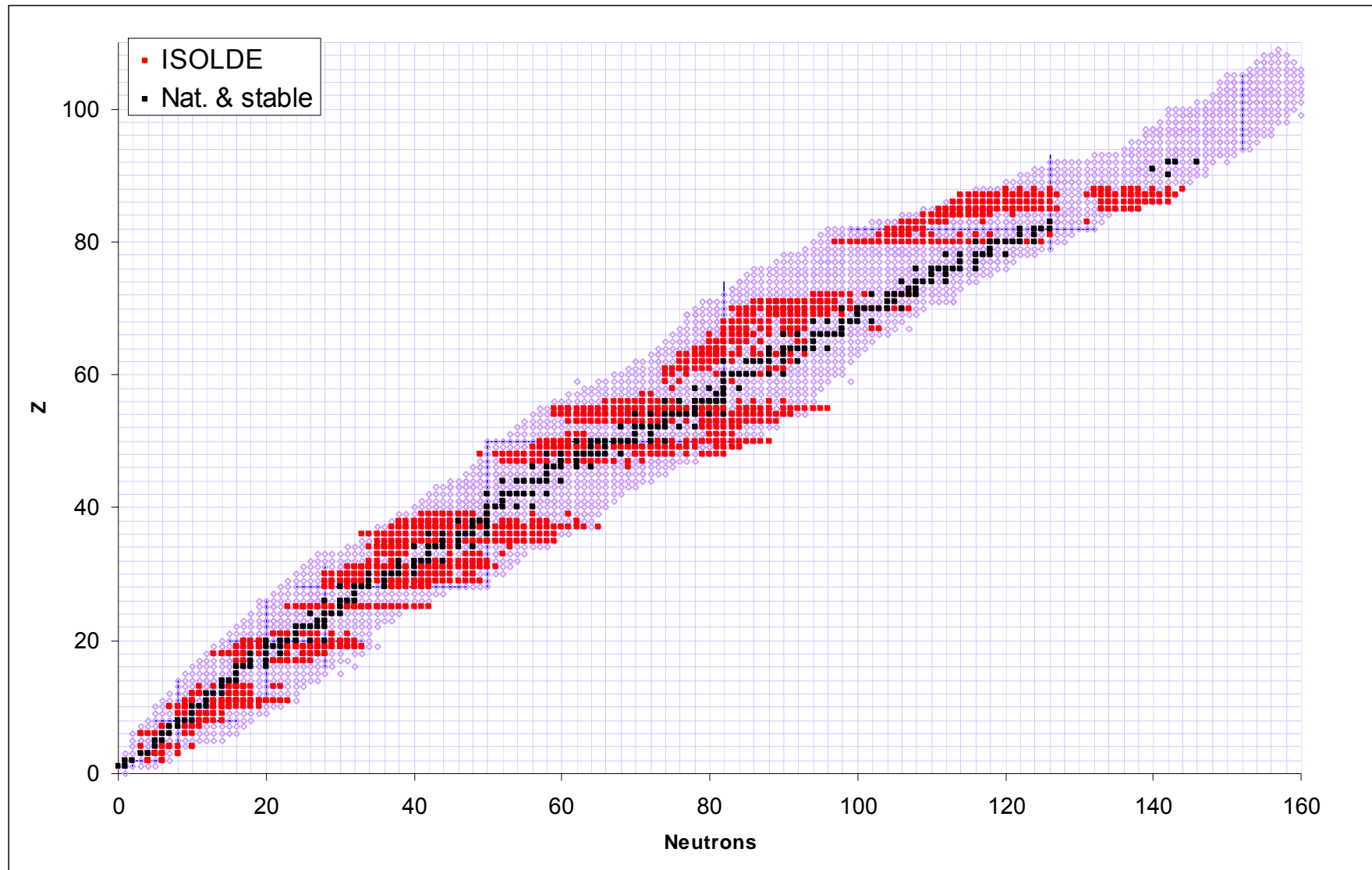
$^{32}\text{Na}$  (13.5 ms)  
 $^{33}\text{Na}$  (8.2 ms)

.....

# Half lives



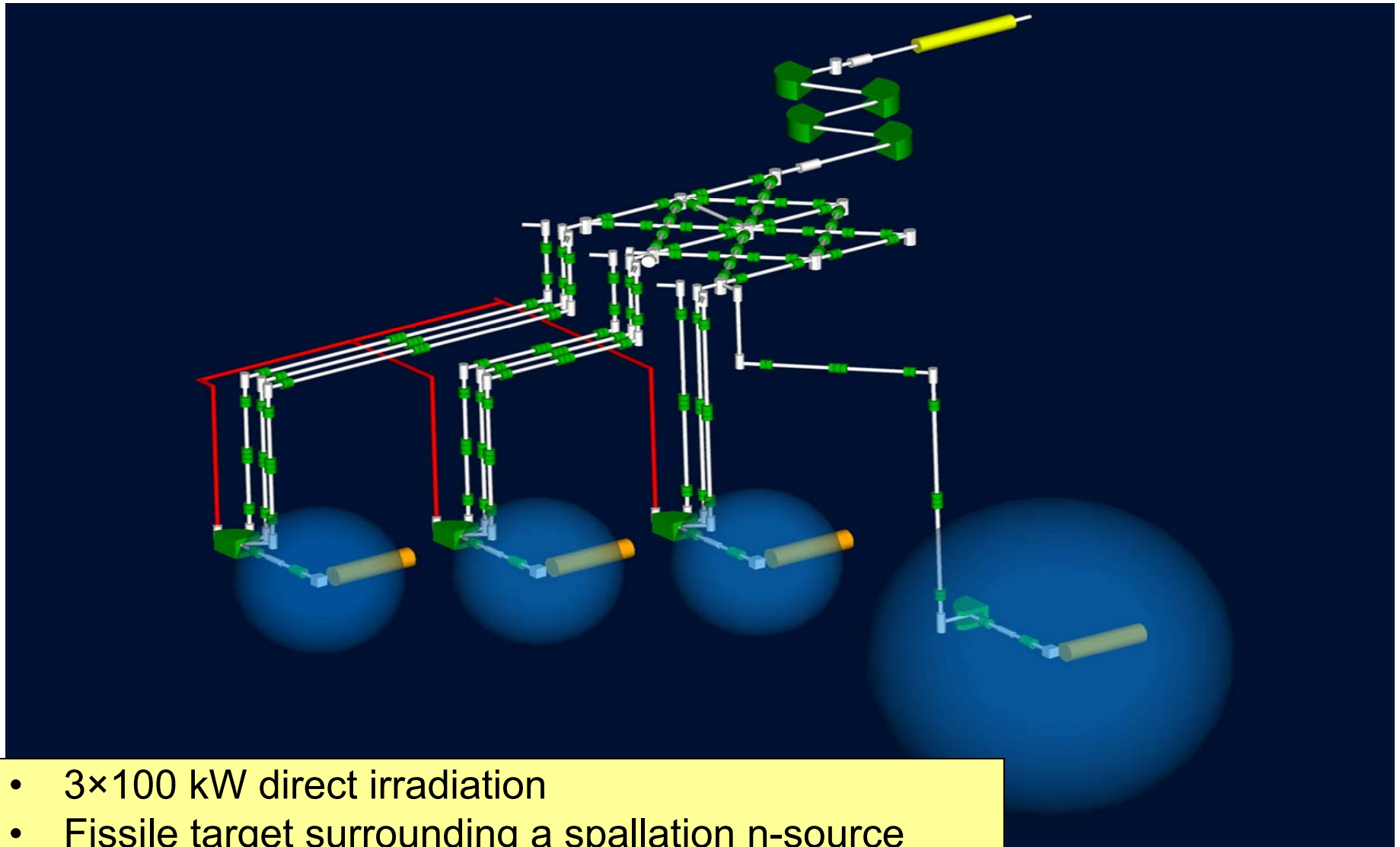
# ISOLDE yields: ~750 RIBs produced (1963-2004)



# EURISOL -- EURISOL DS

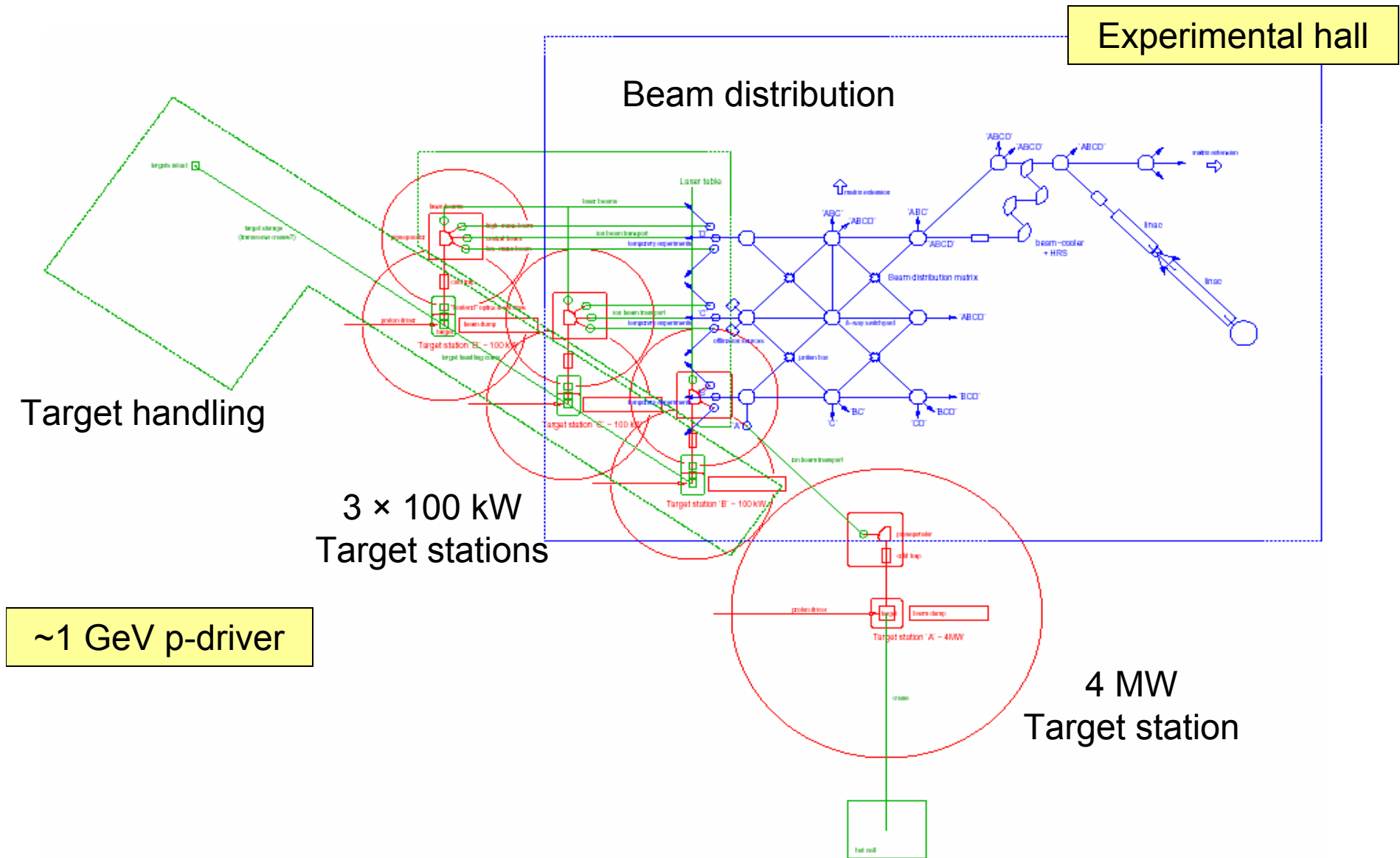
- 4 years EU project published end 2003;
- Conclusion: RIB yields enhancement predicted (vs. 1999 data) by factors of 2 to 4 orders of magnitude !
- 2005 EURISOL-DS to address the remaining technological challenges

# EURISOL target stations



- 3×100 kW direct irradiation
- Fissile target surrounding a spallation n-source
  - >100 kW Solid converters (RAL 800 kW operational)
  - 4 MW Hg-jet

# Top view





# 100 kw direct irradiation

- Target Materials: > **25**
  - Thickness, Heat transport, dose rate
  - Diffusion and Effusion delays
- Chemical nature of the target: **6**
- Oven Materials: **4**
- Elements to be produced: > **70**
- Transfer line: **4**
  - Drift fields
- Ion source: **5**
  - Stabilisation of the production (Std. eff., life time, reproduce on-line the best off-line results).
  - Radiation Hardness.
  - Selectivity vs. isobars.
- Maintenance, vacuum vessel, pumps, radioactive Waste handling.

**100**  
Target and  
Ion-source  
systems

**One**  
Standard  
Front-end ?

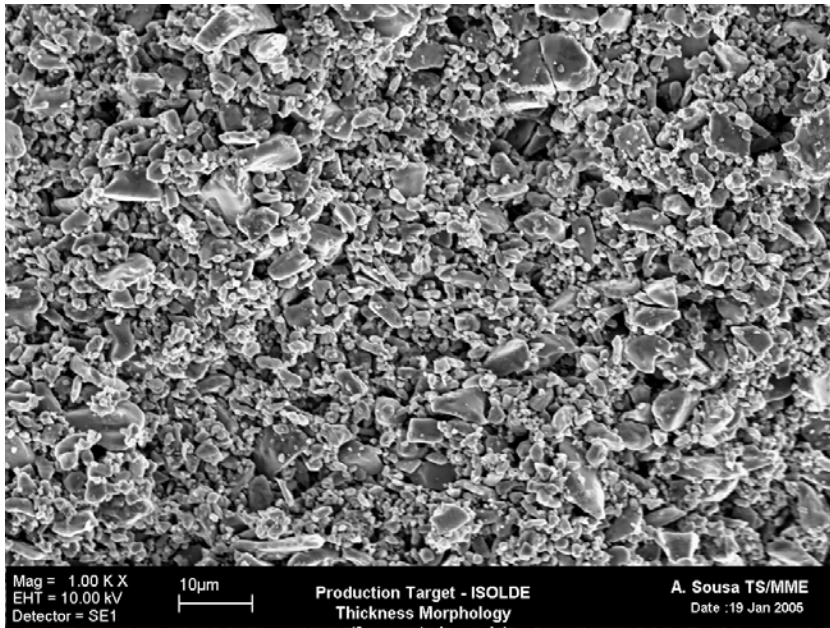
# TEST cases 100 kW direct

- *Targets*
  - Actinide target (**carbide**)
    - UC<sub>2</sub>+C, ThC<sub>2</sub>+C
    - W-converter, Moderator & Reflector
  - Metal foil target (**solid**)
    - Ta, Nb
  - **Oxide** powder of fiber
    - BeO + converter
    - Insulating materials low dE/dx
    - Low density
  - Molten metal (**Liquid**)
    - Vapor condensation
- *Ion-sources*
  - Mono ECR
  - RILIS, Surface
  - FEBIAD
- *Elements*
  - He, Li, Be, Hg ...

4 Targets  
4 Ion-sources  
1 Front-end

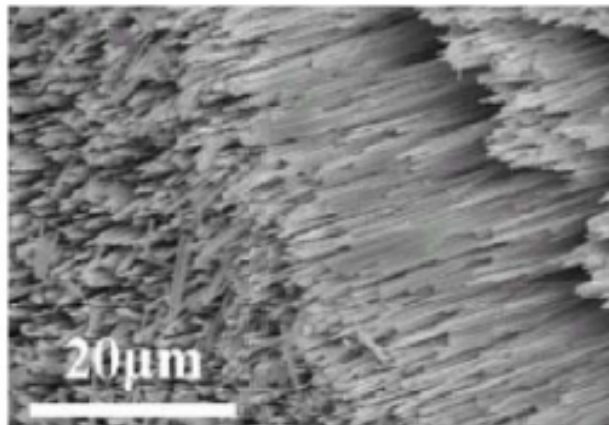
Synergy with  $\beta$ -beam

Spin off is expected on:  
- Similar target materials  
- Elements from the same chemical group

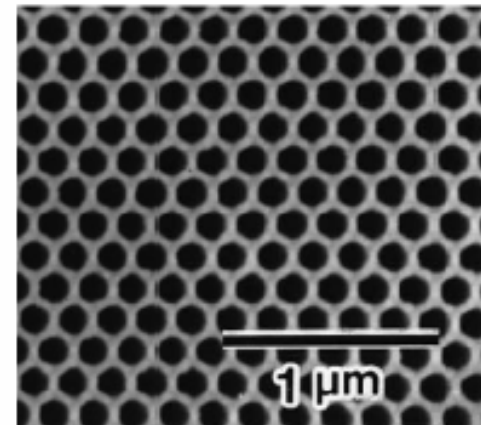


**SiC Triumph**

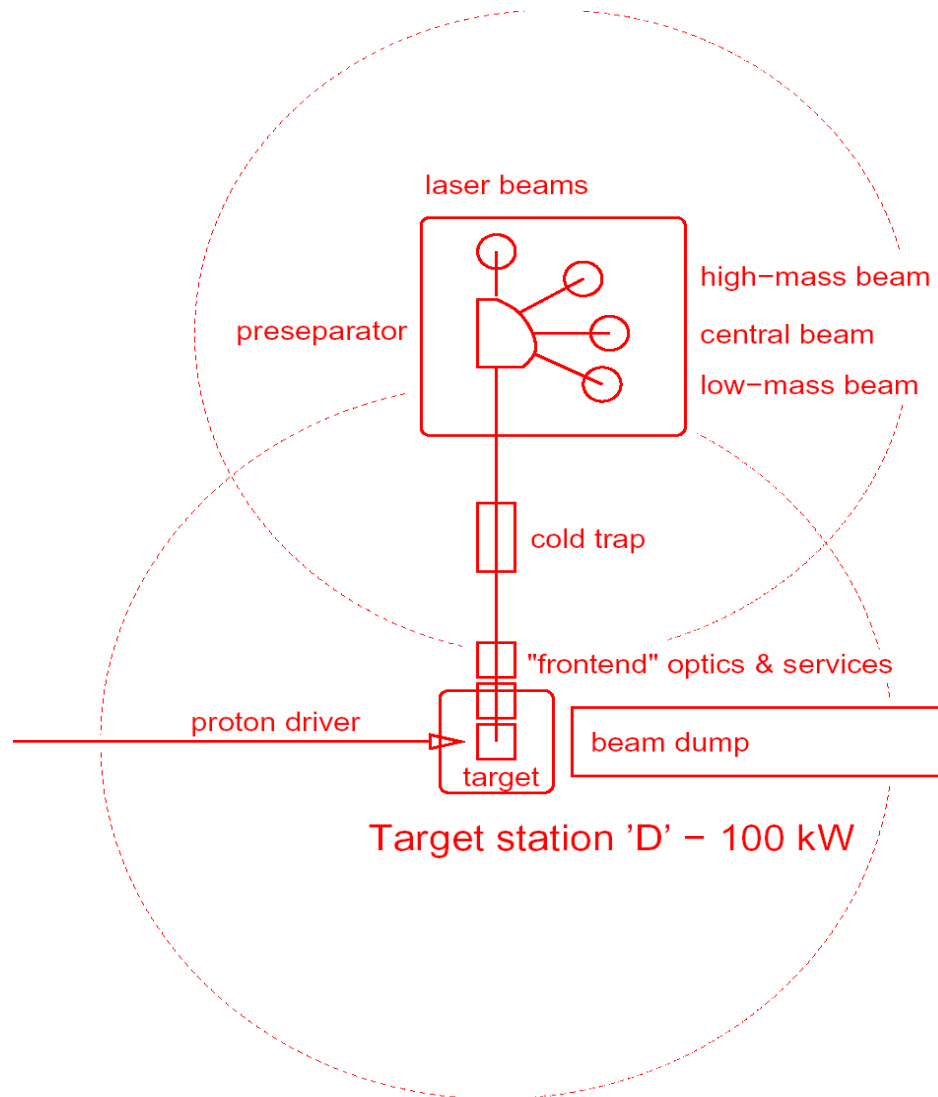
## Nanowires



## Nanoholes



# Confinement of the activity, RIB-selection

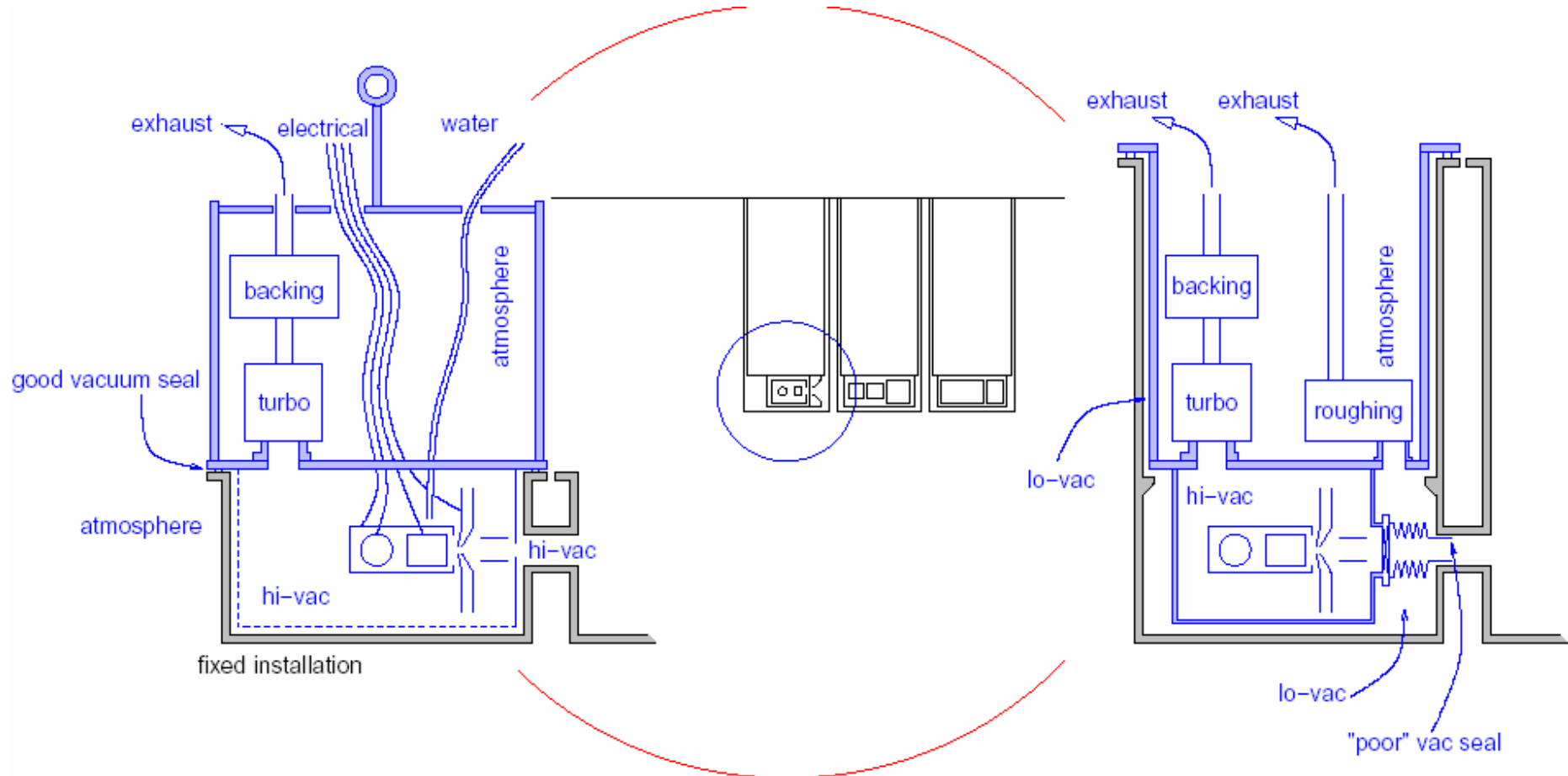


Mass separator

Confinement of  
radioactive gases  
(cold trap)

Target station  
Chemically selective  
ion-source (RILIS)

# Target plugs



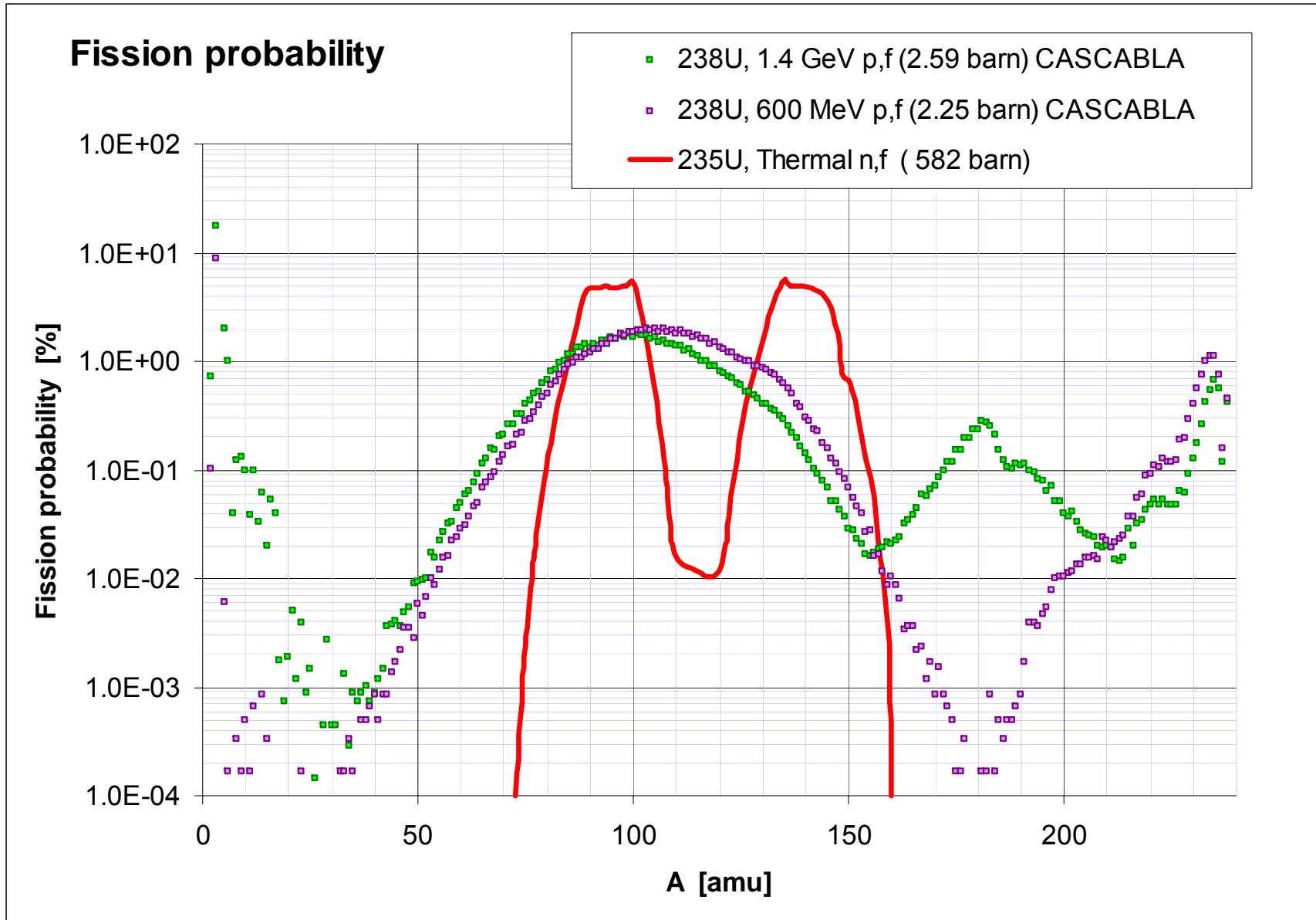
# Actinide target

- Cylindrical, donut or C-shaped optimized vs. n-flux ... and target exchange.
- Target material: Test of the release properties of high density  $UC_2$  vs.  $UC_2+C$  powder
- Thermal equilibrium issue: the target is kept at  $2200^\circ C$  while its inner or close by placed Hg-n-spallation source Has to evacuate  $\sim 1$  MW.

Synergy with SNS, SPIRAL II and SPES  
 Competitive method: high flux reactors  
 $^{235}U$ -fission at MAAF

1 Target ?  
 5 Ion-sources  
 1 Front-end

# p- or n-induced fission

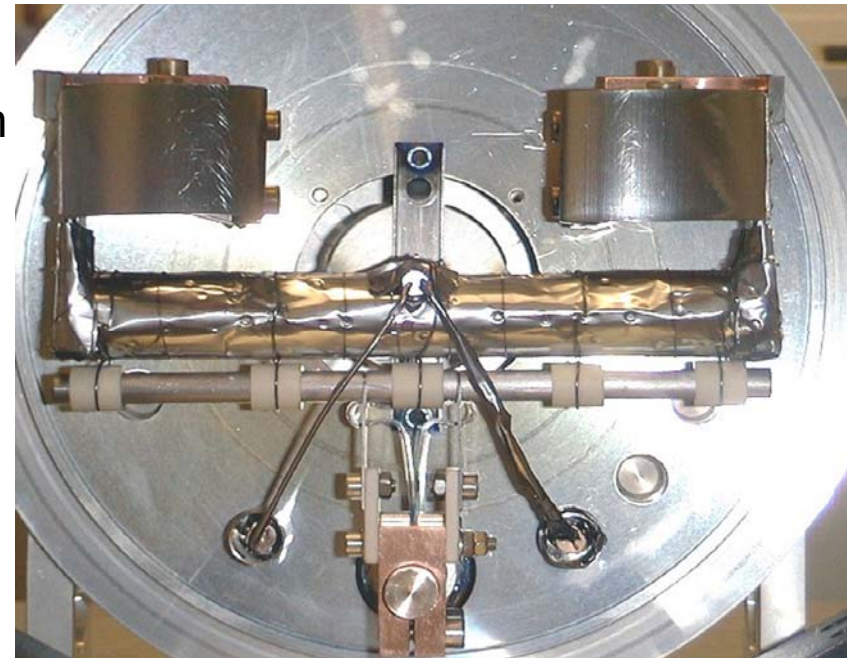




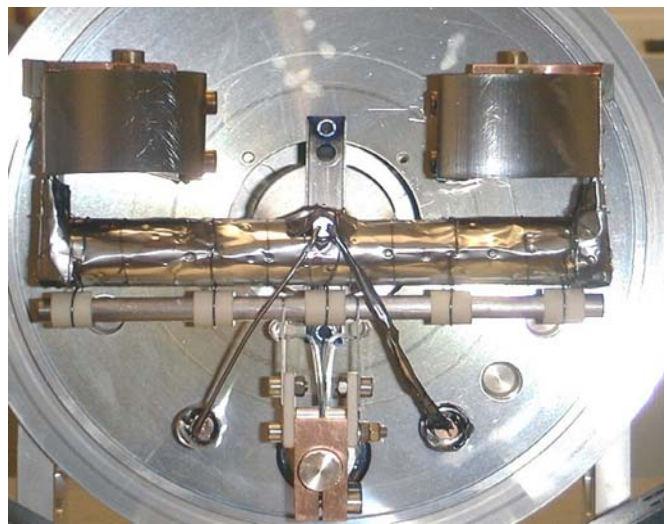
# Spallation n-sources

- Cooled Ta or W-rods
- Hg stream or confined Hg-jet
  - **Engineering study** of the thermal hydraulics, fluid dynamics and construction materials of a window free liquid-metal converter.
  - Study of an innovative **waste management** in the liquid Hg-loop e.g. by means of Hg distillation.
  - Engineering **design and construction** of a functional Hg-loop.
  - Off-line testing and validation of the thermal hydraulics and fluid dynamics.
  - Detailed planning and proposal for subsequent **in-beam test** in collaboration with other Hg target users.
  - Engineering **design of the entire target station** and its handling method

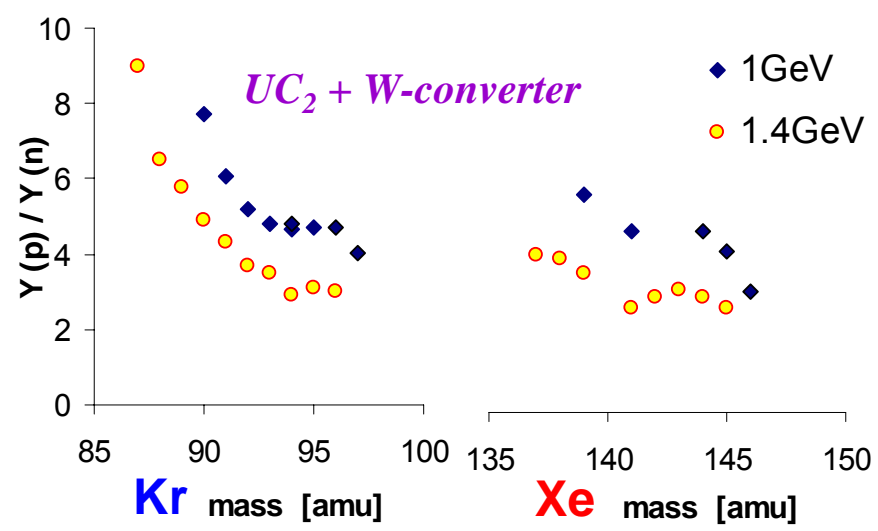
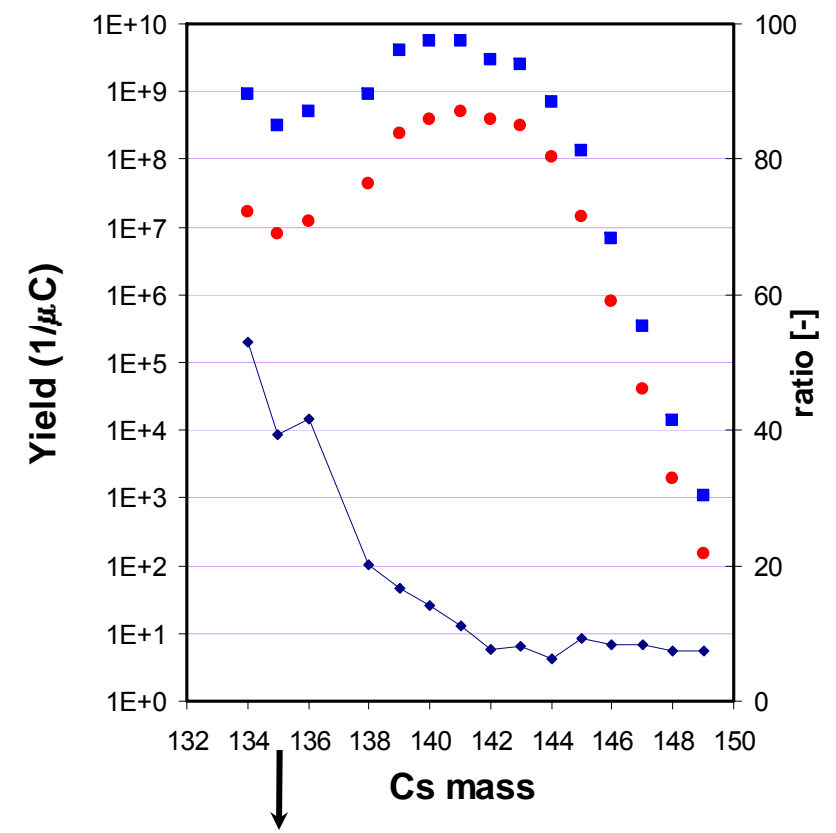
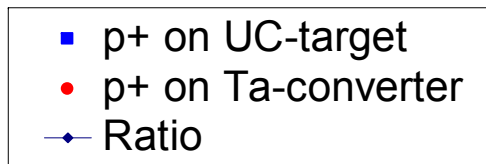
ISOLDE n-spallation source:  
Ta(W)-rod mounted below  
the UC target  
(before irradiation)



# Kr, Xe and Cs yields, Ta-W converter



## Cs-yields UC<sub>2</sub>-183



The yields of very n-rich isotopes obtained via neutron induced fission of Th or U are close to those of high energy protons.

Further developments:

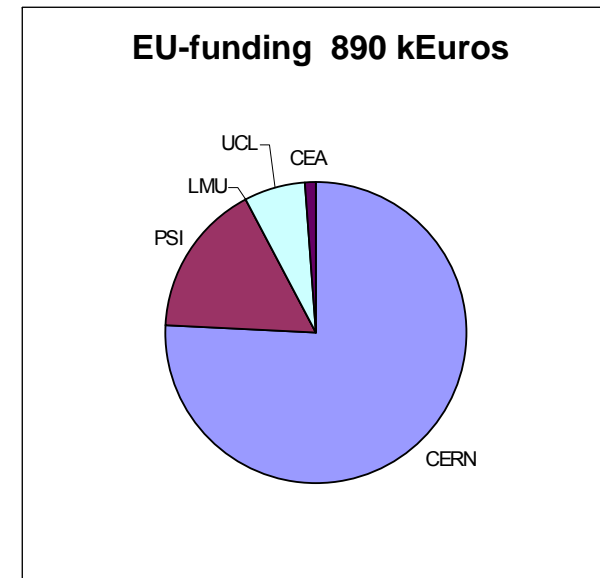
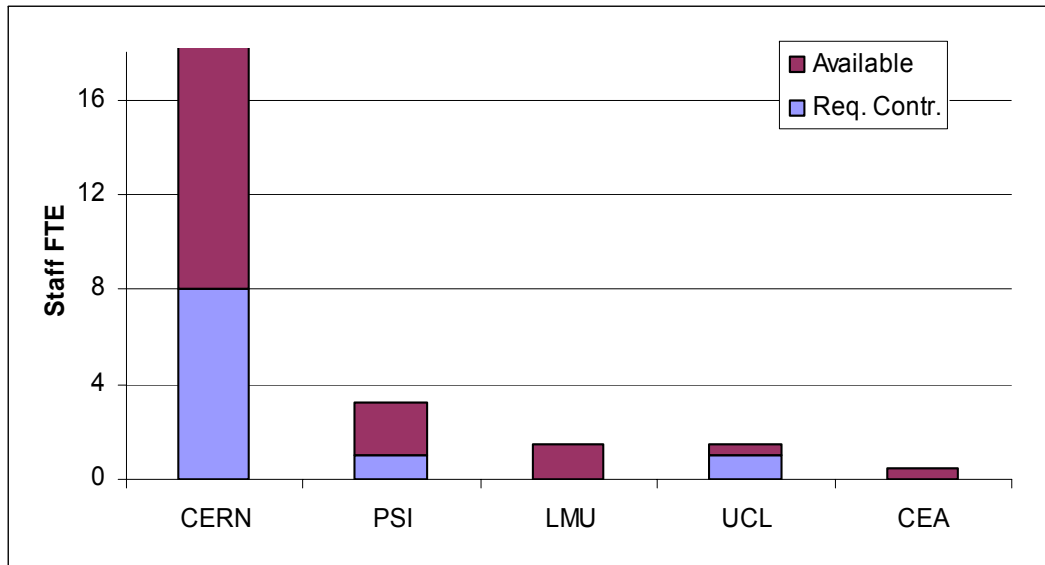
Geometrical optimum and n-reflectors

# Conclusion

- The EURISOL-DS-targetry group proposes:
  - 4 test cases to define the front-end of the 100 kW target stations (T3).
  - Investigation of a spallation n-source based on Hg loop (4 MW) (T2) and cooled solids (<1 MW) (T4).
  - Compare yields of high-density and powder of actinide carbides (T4).
- Decision on EU-funding of the

A stylized logo for EURISOL-DS, with the letters in a multi-colored font (E: red, U: orange, R: yellow, I: green, S: blue, O: purple, L: dark blue, -: grey, D: red, S: blue) and a shadow effect.

# T3 – direct target



**Total estimated Cost (k€): 2439**  
**Contribution from UE (k€): 890**

